

sign language interpreters, must inform the Commission in advance of those needs. Subject to 29 CFR 2706.150(a)(3) and 2706.160(d).

CONTACT PERSON FOR MORE INFO: Jean Ellen, (202) 434-9950, (202) 708-9300 for TDD Relay, 1-800-877-8339 for toll free.

Jean H. Ellen,
Chief Docket Clerk.

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NUCLEAR REGULATORY COMMISSION

Documents Containing Reporting or Recordkeeping Requirements: Office of Management and Budget (OMB) Review

AGENCY: U. S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of the OMB review of information collection and solicitation of public comment.

SUMMARY: The NRC has recently submitted to OMB for review the following proposal for the collection of information under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35).

1. *Type of submission, new, revision, or extension:* Revision.

2. *The title of the information collection:* Proposed Rule to Update 10 CFR part 52, "Early Site Permits; Standard Design Certifications; and Combined Licenses for Nuclear Power Plants."

3. *The form number if applicable:* N/A.

4. *How often the collection is required:* One occasion and every 10 to 20 years for applications for renewal.

5. *Who will be required or asked to report:* Designers of commercial nuclear power plants, electric power companies, and any person eligible under the Atomic Energy Act to apply for a construction permit for a nuclear power plant.

6. *An estimate of the number of responses:* 0.

7. *The estimated number of annual respondents:* 0.

8. *An estimate of the total number of hours needed annually to complete the requirement or request:* 3,429 hours, however, no combined license applications are anticipated during the next three years.

9. *An indication of whether Section 3507(d), Pub. L. 104-13 applies:* N/A.

10. *Abstract:* The NRC is proposing to reorganize 10 CFR part 52 to establish

a separate subpart for each of the seven licensing processes currently described in 10 CFR part 52 (early site permits, early site reviews, standard design certification, standard design approvals, combined licenses, manufacturing licenses, and duplicate design licenses). The purpose of this reorganization is to clarify that each licensing process has equal standing. In addition, several subparts would be reserved for future licensing processes. No substantive changes are intended by the incorporation of current Appendices M, N, O, and Q into the new subparts in 10 CFR part 52.

The NRC is also proposing to retitle 10 CFR part 52 as "Additional Licensing Processes for Nuclear Power Plants," to clarify that the licensing processes in 10 CFR part 52 are in addition to and supplement the two-step licensing process in 10 CFR part 50 and the license renewal process in 10 CFR part 54, and are not limited to the early site permit, standard design certification, and combined license processes as the current title implies.

Submit, by August 15, 2003, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the submittal may be viewed free of charge at the NRC Public Document Room, One White Flint North, 11555 Rockville Pike, Room O-1 F23, Rockville, MD 20852. The proposed rule indicated in "The title of the information collection" is or has been published in the **Federal Register** within several days of the publication date of this **Federal Register** Notice. The OMB clearance package and rule are available at the NRC worldwide Web site: <http://www.nrc.gov/public-involve/doc-comment/Omb/index.html> for 60 days after the signature date of this notice and are also available at the rule forum site, <http://ruleforum.llnl.gov>.

Comments and questions should be directed to the OMB reviewer by August 15, 2003: Bryon Allen, Office of Information and Regulatory Affairs (3150-0151), NEOB-10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be submitted by telephone at (202) 395-3087.

The NRC Clearance Officer is Brenda Jo. Shelton, 301-415-7233.

Dated at Rockville, Maryland, this 10th day of July 2003.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 03-17961 Filed 7-15-03; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-311]

PSEG Nuclear, LLC Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission or the NRC) is considering issuance of an amendment to Facility Operating License No. DPR-75 issued to PSEG Nuclear, LLC (PSEG or the licensee) for operation of the Salem Nuclear Generating Station (Salem), Unit No. 2, located in Salem County, New Jersey.

The proposed amendment would revise License Condition 2.C.10, "Fire Protection," to reflect changes to the Salem post-fire Safe Shutdown (SSD) strategy for Fire Areas 2-FA-AB-64B, 2-FA-AB-84C, and 2-FA-AB-84B. The proposed changes were submitted as a result of PSEG's re-analysis of post-fire SSD capability and recent plant modifications implemented in response to resolution of Electrical Raceway Fire Barrier System issues at Salem.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in title 10 of the Code of Federal Regulations (10 CFR), section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a

margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed changes to the Salem Unit 2 post-fire safe shutdown (SSD) in fire areas 2-FA-AB-64B, 2-FA-AB-84C and 2-FA-AB-84B only impact Salem Unit No. 2's response in the event of a fire. No other design basis events are impacted by the proposed changes. These proposed changes do not increase the probability of fire event that has been previously analyzed. The likelihood of fire event is not increased since the proposed change does not alter the fire hazards contained in the plant. []

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed change to [the] post-fire SSD strategy in fire areas 2-FA-AB-64B, 2-FA-AB-84C and 2-FA-AB-84B does not create the possibility of a new or different kind of accident. The design basis event applicable to this proposal is that of a fire event in the three subject fire areas, therefore a new or different kind of accident is not introduced. [T]he revised SSD strategy ensures that Salem Unit 2 can be safely shutdown in the event of a fire in these areas.

Therefore, this proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No.

The proposed changes to the post-fire SSD strategy for fire areas 2-FA-AB-64B, 2-FA-AB-84C and 2-FA-AB-84B do not reduce the margin of safety in response to a fire in these areas. The proposed deviations from 10 CFR 50 Appendix R Section III.G.3 and III.L.3 do not impede Salem Unit 2's ability to safely shutdown in the event of a fire in these areas. Modifying the plant to comply with these requirements would not significantly increase the margin of safety in the event of fire in these areas. The changes to the post-fire SSD strategy in these areas along with the modifications performed to support these changes ensure that a level of margin of safety is maintained.

As a result, this change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish, in the **Federal Register**, a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By August 15, 2003, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR part 2. Interested persons should consult a current copy of 10 CFR 2.714, which is available at the Commission's

Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, or electronically on the Internet at the NRC Web site <http://www.nrc.gov/reading-rm/doc-collections/cfr/>. If there are problems in accessing the document, contact the Public Document Room Reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth, with particularity, the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted, with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene, or who has been admitted as a party, may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific

sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. The petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing, or a petition for leave to intervene, must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland, by the above date. Because of the continuing disruptions in delivery of mail to United States Government offices, it is requested that petitions for leave to intervene and requests for hearing be transmitted to the Secretary of the Commission either by means of facsimile transmission to 301-415-1101 or by e-mail to hearingdocket@nrc.gov. A copy of the petition for leave to intervene and request for hearing should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory

Commission, Washington, DC 20555-0001, and because of continuing disruptions in delivery of mail to United States Government offices, it is requested that copies be transmitted either by means of facsimile transmission to 301-415-3725 or by e-mail to OGCMailCenter@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to Jeffrie J. Keenan, Esquire, Nuclear Business Unit—N21, P.O. Box 236, Hancocks Bridge, NJ 08038, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated July 1, 2003, which is available for public inspection at the Commission's PDR, located at One White Flint North, File Public Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records can be accessible from the Agencywide Documents Access and Management System's (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS, or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 9th day of July 2003.

For the Nuclear Regulatory Commission.

Robert J. Fretz,

Project Manager, Section 2, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 03-17959 Filed 7-15-03; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-413 and 50-414]

Duke Energy Corporation, North Carolina Electric Membership Corporation, Saluda River Electric Cooperative, Inc., Catawba Nuclear Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of exemptions from Title 10 of the Code of Federal Regulations (10 CFR) part 50, section 50.44, section 50.46, and Appendix K, for Facility Operating License Nos. NPF-35 and NPF-52, issued to Duke Power Company, et al, (the licensee), for operation of the Catawba Nuclear Station (CNS), Units 1 and 2, located in York County, South Carolina. Therefore, pursuant to 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt the Catawba Nuclear Station, Units 1 and 2, from the requirements of 10 CFR 50.44, 10 CFR 50.46 and 10 CFR part 50, Appendix K, to allow the use of eight Lead Test Assemblies (LTAs) fabricated with a cladding material that contains a nominally lower tin content than previously approved cladding materials.

The proposed action is in accordance with the licensee's application dated December 3, 2002, as supplemented by letter dated April 8, 2003.

The Need for the Proposed Action

As the nuclear industry pursues longer operating cycles with increased fuel discharge burnups and more aggressive fuel management, the corrosion performance specifications for the nuclear fuel cladding become more demanding. Industry data indicates that corrosion resistance improves for cladding with a lower tin content. The optimum tin level provides a reduced corrosion rate while maintaining the benefits of mechanical strengthening and resistance to accelerated corrosion from abnormal chemistry conditions. In addition, fuel rod internal pressures (resulting from the increased fuel duty, use of integral fuel burnable absorbers and corrosion/temperature feedback effects) have become more limiting with respect to fuel rod design criteria. By reducing the associated corrosion buildup, and thus, minimizing temperature feedback effects, additional