

and Technology, Institute of Museum and Library Services, 1100 Pennsylvania Ave., NW., Room 223, Washington, DC 20506.

SUPPLEMENTARY INFORMATION:

I. Background

The Institute of Museum and Library Services is an independent Federal grant-making agency authorized by the Museum and Library Services Act, Public Law 104-208. The IMLS provides a variety of grant programs to assist the nation's museums and libraries in improving their operations and enhancing their services to the public. Museums and libraries of all sizes and types may receive support from IMLS programs. In the National Leadership Grant Program, IMLS funds the digitization of library and museum collections.

This study is to determine the feasibility of using the Open Archives Initiative (OAI) Metadata Harvesting Protocol to aggregate and provide integrated item-level search access to the digitization projects funded by the Institute of Museum and Library Services through the National Leadership Grant Program.

II. Current Action

To develop an effective Open Archives Initiative metadata harvesting protocol information will be collected from focus groups of resource developers.

Agency: Institute of Museum and Library Services

Title: Study of IMLS Funded Digital Collections and Content

OMB Number: none.

Agency Number: 3137.

Frequency: Once.

Affected Public: Museums and libraries that created digital collections with IMLS funding.

Number of Respondents: 20, in two focus groups.

Estimated Time Per Respondent: 1.5 hours.

Total Burden Hours: 30.

Total Annualized capital/startup costs: n/a.

Total Costs: \$1,125.

FOR FURTHER INFORMATION CONTACT:

Comments should be sent to the Office of Information and Regulatory Affairs, Attn.: OMB Desk Officer for Education, Office of Management and Budget, Room 10235, Washington, DC 20503 (202) 395-7316.

Dated: December 17, 2003.

Rebecca Danvers,

Director, Research and Technology.

[FR Doc. 03-31518 Filed 12-22-03; 8:45 am]

BILLING CODE 7036-01-M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-305]

Nuclear Management Company, LLC, Kewaunee Nuclear Power Plant; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment to Facility Operating License No. DPR-43, issued to Nuclear Management Company, LLC (the licensee), for operation of the Kewaunee Nuclear Power Plant, located in Kewaunee County, Wisconsin. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would revise the Kewaunee Nuclear Power Plant operating license and technical specifications (TSs) to increase the licensed rated power by 6.0 percent from 1673 megawatts thermal (MWt) to 1772 MWt.

The proposed action is in accordance with the licensee's application dated May 22, 2003.

The Need for the Proposed Action

The proposed action permits an increase in the licensed core thermal power from 1673 MWt to 1772 MWt for the Kewaunee Nuclear Power Plant, providing the flexibility to obtain a higher electrical output from the Kewaunee Nuclear Power Plant with minimal modifications.

Environmental Impacts of the Proposed Action

The licensee has submitted an environmental evaluation supporting the proposed stretch power uprate and provided a summary of its conclusions concerning the radiological and non-radiological environmental impacts of the proposed action.

Radiological Environmental Assessment

The stretch power uprate will increase the activity level of radioactive isotopes in the primary and secondary coolant. Due to leakage or process operations, fractions of these fluids are transported to the liquid and gaseous radwaste systems where they are processed prior to discharge. As the activity levels in the primary and secondary coolant are increased, the activity level of radwaste inputs is

proportionately increased. Regulatory guidance relative to methodology to be utilized to establish whether the radwaste effluent releases from a pressurized-water reactor meet the requirements of 10 CFR part 20 and 10 CFR part 50, appendix I, is provided in NUREG-0017, Revision 1. The NUREG-0017 methodology is independent of the length of the fuel cycle.

The maximum expected increase in the reactor coolant source (associated with the chemical group with the largest percentage increase) is approximately 17.6 percent for noble gas activity. This increase is primarily a combination of the impact of core power uprate and reduction in reactor coolant system (RCS) mass. Considering the accuracy and error bounds of the operational data utilized in NUREG-0017, this percentage change is well within the uncertainty of the existing NUREG-0017-based expected reactor coolant isotopic inventory used for radwaste effluent analyses and corrected for a facility with this power rating.

As discussed above, there is approximately a 17.6 percent increase assumed for the liquid releases as input activities are based on the largest long-term RCS activity increase for any chemical grouping and on waste volumes which are essentially independent of power level within the applicability range of NUREG-0017. Tritium releases in liquid effluents are assumed to increase approximately 11.4 percent (corresponding to the effective increase in core power) since the facility is changing its power rating, without changing its operational procedures. However, for all liquid releases, the power uprate analysis conservatively used the worst case scaling factor for all isotopes between the pre-uprate case and the uprate case.

For all noble gases (limiting chemical group), there will be a maximum 17.6 percent increase in effluent releases due to the core power uprate. Gaseous releases of Kr-85 in actuality will increase by approximately 11.4 percent. Isotopes with shorter half lives will have increases slightly greater than the percentage increase in power level. The decrease in RCS mass (approximately 5 percent) contributes to the increased concentration of this chemical group in the RCS (the primary removal term for the non-Kr-85 noble gases is decay in the RCS) such that the impact of power uprate is conservatively approximated at 17.6 percent. The impact of the power uprate in iodine releases is approximated by the power level increase. The other components of the gaseous release (that is, particulates via the building ventilation systems and

water activation gases) are not impacted by the power uprate using the methodology outlined in NUREG-0017. Tritium releases in the gaseous effluents increase in proportion to the increased tritium production, which is directly related to core power and is pathway allocated in the analysis in the same ratio as pre-power uprate releases. For particulates, the methodology of NUREG-0017 specifies the release rate per year per unit per building ventilation system. This is not dependent on power level. Thus, there is no change calculated for the power uprate. However, a 17.6 percent increase will be conservatively addressed.

The maximum increase in doses for gaseous and liquid effluents is estimated to be 17.6 percent. The estimated doses are a very small fraction of that allowable under appendix I.

Only minor, if any, changes in waste generation volume are expected. However, it is expected that the activity levels for most of the solid waste would increase proportionately to the increase in long half life coolant activity. Thus, while the total longlived activity contained in the waste is expected to be bounded by the percentage of the power uprate, the increase in the overall volume of waste generation resulting from the power uprate is expected to be minor.

The licensee stated that the power uprate has no significant impact on the expected annual radwaste effluent releases or doses (that is, all doses remain a small percentage of allowable Appendix I doses). The licensee concluded that following the power uprate, the liquid and gaseous radwaste effluent treatment system will remain capable of maintaining normal operation offsite doses within the requirements of 10 CFR Part 50, Appendix I.

Dose Consideration

The stretch power uprate will impact the radiation source terms in the core and the expected radiation source terms in the coolant. The actual increase in radiation levels due to the power uprate will not significantly affect radiation zoning or shielding requirements in the various areas of the plant because it is expected that the increase due to the power uprate will be offset by (1) the conservative analytical techniques typically used to establish shielding requirements, (2) the conservatism in the pre-power uprate design-basis RCS source terms used to establish the radiation zones, and (3) the plant TSs that limit the RCS concentrations to levels well below the design-basis source terms. Individual worker

exposures will be maintained within acceptable limits by the site as-low-as- reasonably-achievable program that controls access to radiation areas. The licensee stated that the stretch power uprate has no significant effect on normal plant operation radiation zones and shielding requirements.

Following the power uprate, the licensee stated that the post-LOCA vital area operator dose estimates will remain within the regulatory limits of NUREG-0578, Item 2.1.6.b and NUREG-0737 II.B.2 and II.B.3.

Non-Radiological Environmental Assessment

The licensee assessment included determining whether the power uprate will cause the plant to exceed the National Pollutant Discharge Elimination System (NPDES) permits' effluent discharge limitations and other conditions associated with operation of the plant. This review is based upon information contained in the State of Wisconsin, Department of Natural Resources (WDNR), WPDES Permit No. WI-0001571-06-0 and the Final Environmental Statement for the Kewaunee Nuclear Power Plant. The WPDES permit was effective beginning on August 1, 2002, and expires June 30, 2005. The licensee stated that there are no requirements in the NPDES Permit impacted by the power uprate. Circulating water outlet temperature rise increases by approximately 1.5 °F due to the power uprate. The total temperature rise across the condenser would be 16.7 °F. No change in the circulating water flow is required due to the power uprate. The 1.5 °F increase in the circulating water outlet temperature rise is acceptable because it is within the licensee's WPDES Permit No. WI-0001571-06-0.

Summary

The NRC has completed its evaluation of the proposed action and concludes that there are no significant environmental impacts associated with the proposed action.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not have a potential to affect any historic sites. It has a small affect on

nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (*i.e.*, the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resource than those previously considered in the Final Environmental Statement for the Kewaunee Nuclear Power Plant, dated December 1972.

Agencies and Persons Consulted

On November 4, 2003, the staff consulted with the Wisconsin State official, Jeff Kitzebul of the Public Service Commission—Electric Division, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated May 22, 2003. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-

397-4209 or 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 16th day of December 2003.

For the Nuclear Regulatory Commission.

L. Raghavan,

*Chief, Section 1, Project Directorate III,
Division of Licensing Project Management,
Office of Nuclear Reactor Regulation.*

[FR Doc. 03-31577 Filed 12-22-03; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Sunshine Notice

AGENCY: Nuclear Regulatory Commission.

DATES: Weeks of December 22, 29, 2003, January 5, 12, 19, 26, 2004.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and closed.

MATTERS TO BE CONSIDERED:

Week of December 22, 2003

There are no meetings scheduled for the Week of December 22, 2003.

Week of December 29, 2003—Tentative

There are no meetings scheduled for the Week of December 29, 2003.

Week of January 5, 2004—Tentative

There are no meetings scheduled for the Week of January 5, 2004.

Week of January 12, 2004—Tentative

Wednesday, January 14, 2004

9:30 a.m. Briefing on Status of Office of Chief Information Officer Programs, Performance, and Plans (Public Meeting) (Contact: Jacqueline Silber, 301-415-7330)

This meeting will be webcast live at the Web address <http://www.nrc.gov>.

Week of January 19, 2004—Tentative

Wednesday, January 21, 2004

1:30 p.m. Discussion of Security Issues (Closed—Ex. 1)

Week of January 26, 2004—Tentative

There are no meetings scheduled for the Week of January 26, 2004.

The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (recording)—(301) 415-1292.

Contact person for more information: Timothy J. Frye, (301) 415-1651.

Additional Information: By a vote of 3-0 on December 17, the Commission determined pursuant to U.S.C. 552b(e)

and § 9.107(a) of the Commission's rules that "Affirmation of (1) SECY-03-0195 (Final Rule: 10 CFR Part 50, Financial Information Requirements for Applications to Renew or Extend the Term of an Operating License for a Power Reactor); and (2) SECY-03-0211 (Dominion Nuclear Connecticut, Inc., Millstone Nuclear Power Station, Unit 2)" be held on December 18, and on less than one week's notice to the public.

The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/what-we-do/policy-making/schedule.html>.

This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to the distribution, please contact the Office of the Secretary, Washington, DC 20555, (301) 415-1969. In addition, distribution of this meeting notice over the Internet system is available. If you are interested in receiving this Commission meeting schedule electronically, please send an electronic message to dkw@nrc.gov.

Dated: December 18, 2003.

Timothy J. Frye,

Technical Coordinator, Office of the Secretary.

[FR Doc. 03-31669 Filed 12-19-03; 11:02 am]

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NUCLEAR REGULATORY COMMISSION

Biweekly Notice; Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations

Background

Pursuant to Public Law 97-415, the U.S. Nuclear Regulatory Commission (the Commission or NRC staff) is publishing this regular biweekly notice. Public Law 97-415 revised section 189 of the Atomic Energy Act of 1954, as amended (the Act), to require the Commission to publish notice of any amendments issued, or proposed to be issued, under a new provision of section 189 of the Act. This provision grants the Commission the authority to issue and make immediately effective any amendment to an operating license upon a determination by the Commission that such amendment involves no significant hazards consideration, notwithstanding the pendency before the Commission of a request for a hearing from any person.

This biweekly notice includes all notices of amendments issued, or proposed to be issued from November 27 through December 11, 2003. The last

biweekly notice was published on December 9, 2003 (68 FR 68654).

Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The Commission has made a proposed determination that the following amendment requests involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The basis for this proposed determination for each amendment request is shown below.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received before action is taken. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays.